



Jordan University of Science and Technology
Faculty of Engineering
Nuclear Engineering Department

NE340 Nuclear Reactors Theory

First Semester 2018-2019

Course Catalog

3 Credit Hours. Review on neutron interactions, center of mass system, differential cross-section, compound nucleus formation model, optical model, theory of total neutron cross-section, Doppler broadening, neutron current and flux, neutron transport theory, one-speed neutron diffusion theory, neutron moderation in hydrogenous and non-hydrogenous medium, slowing down density, conditions for criticality of nuclear reactors, Four and six-factor formula, neutrons economy, infinite reactor, reflector saving, and boundary conditions.

Text Book

Title	Nuclear Reactor Analysis
Author(s)	J.J. Duderstadt and L.J. Hamilton
Edition	1st Edition
Short Name	Ref #1
Other Information	John Wiley & Sons, 1976

Course References

Short name	Book name	Author(s)	Edition	Other Information
Ref #2	Introduction to Nuclear Reactor Theory	John R. Lamarsh	1st Edition	Addison-Wesley, 1966

Instructor

Name	Dr. Khaled AL-Shboul
Office Location	E2 L-2
Office Hours	Sun : 12:00 - 13:00 Mon : 13:00 - 16:00 Tue : 12:00 - 13:00 Wed : 13:30 - 14:30
Email	kfshboul@just.edu.jo

Class Schedule & Room
Section 1: Lecture Time: Mon, Wed : 11:30 - 13:00 Room: E2113

Prerequisites		
Line Number	Course Name	Prerequisite Type
2002060	NE206 Introduction To Nuclear Engineering	Prerequisite / Pass

Tentative List of Topics Covered		
Weeks	Topic	References
Week 1	An Introduction to Nuclear Power Generation	Ch 01 From Ref #1
Weeks 2, 3	The Nuclear Physics of Fission Chain Reactions	Ch 02 From Ref #1
Weeks 4, 5, 6	Neutron moderation with and without absorption	Ch 08 From Ref #1 , Ch 06 & Ch 07 From Ref #2
Weeks 6, 7, 8	Fission Chain Reactions and Nuclear Reactors	Ch 03 From Ref #1
Weeks 9, 10, 11	Neutron Transport	Ch 04 From Ref #1
Weeks 12, 13, 14, 15	The One-Speed Diffusion Theory Model	Ch 05 From Ref #1

Mapping of Course Outcomes to Program Student Outcomes	Course Outcome Weight (Out of 100%)	Assessment method
To demonstrate a fundamental understanding of the center of the mass system and differential scattering crosssections. [11]	15%	First Exam
To understand the four and six factors formulas used to determine the reactor multiplication factor. [11]	15%	First Exam
To demonstrate a solid understanding of fundamental transport concepts such as neutron density, neutron scalar flux, neutron current density. [11]	20%	Second Exam
To understand how to calculate neutron interaction probabilities density. [11]	10%	Second Exam
To be able to derive the neutron transport equation and the neutron diffusion equation. [11]	15%	Final Exam
To understand how to solve the one group diffusion equations for certain core geometries. [11, 12]	25%	Final Exam

Relationship to Program Student Outcomes (Out of 100%)						
1	2	3	4	5	6	7
87.50	12.50					

Evaluation	
Assessment Tool	Weight
First Exam	30%
Second Exam	30%
Final Exam	40%

Date Printed: 2020-01-03